

ACTINIDES FILTERING OUT IN THE DEMO-IMITATION SEPARATOR WITH A MAGNETIC FIELD OF A GIVEN CONFIGURATION

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To simulate spent nuclear fuel (SNF) cleaning up from fission products, the concept of a demonstration-imitation separator (DIS) with a plasma rotating in crossed electric and magnetic fields is being clarified. The parameters of a magnetic system consisting of a combination of superconducting and water-cooled windings are presented. The features of plasma drift in a magnetic field of a given geometry are pointed. The estimation of energy consumption for the creation of a magnetic field of the separator, which is ~ 250 kW, has been carried out. A method for removing actinides from a plasma separator into an external container is proposed.

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INTRODUCTION

Earlier, in order to purify the plasma from impurities, the motion of plasma in magnetic fields of various configurations was investigated. In [1], a method is considered for the selection of heated ions in a curvilinear magnetic field for processing spent nuclear fuel (SNF) and radioactive waste. In the given geometry of the magnetic field, the plasma drift is associated with the deviation of ions from the axis of the system in the direction of the binormal to the magnetic field lines. In [2, 3], the mechanisms of purification of hydrogen plasma from impurities during drift in the magnetic field of a toroidal solenoid are considered. In particular, using the magnetic field of a toroidal solenoid, under experimental conditions [2], the possibility of cleaning a hydrogen plasma bunch from impurities located in the slow part of the bunch has been shown. In [4], the motion of a plasma in a toroidal plasma channel with a voltage applied to its wall is considered, and the features of the passage of high-energy and low-energy particles in it are investigated. In [5], in order to separate actinides and lanthanides, it is proposed to use an asymmetric magnetic trap to control the motion of heavy and light ions in a centrifugal magnetic mass filter. In [6], a conceptual design of the demonstration-imitation separator (DIS) for simulating SNF clean up from fission products in a plasma rotating in crossed electric and magnetic fields is presented. A non-radioactive mixture of oxides that imitate SNF is used as a working material. The main component is uranium-238 dioxide. In this paper, we consider the configuration of the magnetic field of the DIS setup, which provides spatial separation of the actinide ions in a plasma from the ions of other type, collection of target ions ($M = 232\dots277$) in a localized zone and a possible method of actinides removing from the separator into an external container.

1. PARAMETERS OF THE DIS MAGNETIC SYSTEM

The magnetic field of the DIS setup is developed using a combined system of windings: superconducting – 1, 2 and water-cooled – 3, 4 (Fig. 1). It is assumed that the working mixture of oxides simulating SNF enters the plasma source (PS) in the form of a micropowder.

Controlling the plasma flux under conditions of magnetization of heavy ions, in particular actinides, requires the creation of strong magnetic fields. For this purpose, it is proposed to use a magnetic system in the PS, which includes two superconducting solenoids with a constructive current density of $\sim 10^4$ A/cm². The geometry of the magnetic field in the PS affects the process of plasma formation and ensures its injection into the vacuum chamber of the plasma mass filter. In our case, a mirror-type magnetic system is created in the PS, formed by windings 1 and 2, with the maxima $B(z) \sim 3.2$ T and $B(z) \sim 2.5$ T, respectively (Fig. 2). Creating a minimum magnetic field is provided by the selection of the current values of solenoids 1 and 2 and their location. A high degree of plasma ionization is achieved due to oscillations of electrons trapped in magnetic field and increasing the number of particles collisions in the plasma ($n_e \sim 10^{14}$ cm⁻³, $T_e \sim 5$ eV). Earlier [7], the influence of the mirror ratio of the magnetic field on the parameters of the plasma formed in the uranium-lanthanum system was studied. In the magnetic system of the PS, the mirror ratio is 2.07 (see Fig. 2), which, according to [7], insignificantly affects the formation of plasma upon ionization of the working mixture containing $\sim 95\%$ of actinide oxides by low-energy electrons (up to 10 eV). An important role for trapping particles in the PS is played by the reflecting wall, where the outlet is located.

In the PS, the magnetized plasma components, moving along the force lines of an inhomogeneous axisymmetric magnetic field, are transported to the outlet ($r = 5$ cm). They include magnetized ions ($M = 232\dots277$), which are injected from the PS in the axial direction into the vacuum chamber of the separator.

In a zone between the coils 2 and 3 there is a strong decrease in the magnetic field, causing the appearance of the longitudinal component of the electric field E_z , the equipotentials of which are the magnetic field lines [4] and the motion of the plasma flux towards the coaxial electrodes, which create a radial electric field, E_r . As a result, the plasma drifts in a decreasing magnetic field at the transport of heavy ions to the separation area with a relatively weak magnetic field (see Fig. 2).

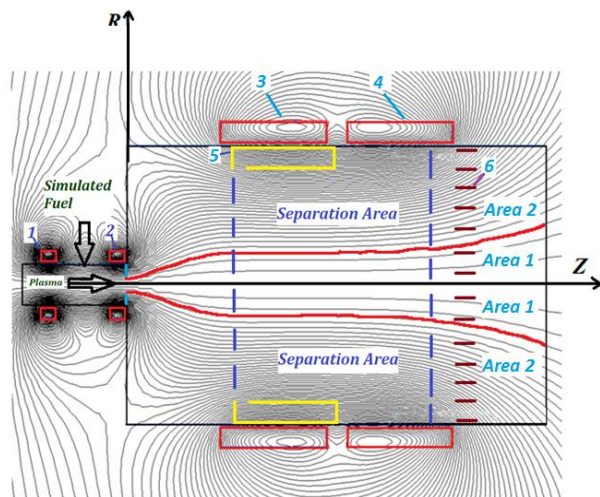


Fig. 1. The magnetic field lines created by the system of solenoids 1, 2, 3, 4 in the DIS setup; 5 – collector of actinides; 6 – coaxial electrodes for creating a radial electric field. The dotted lines are the boundaries of the separation area

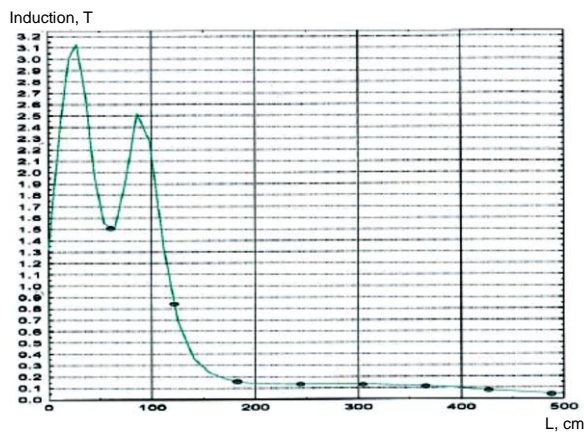


Fig. 2. Axial magnetic field distribution in the DIS setup

The induction (B) of magnetic field in the separator is determined by the axial and radial projections $B_r = B(r)$ and $B_z = B(z)$ in the plane $(r \times z)$. The B_z value is one of the parameters that affects the mass separation of ions in a plasma mass filter. In the separation area, where B_z is constant and the plasma is collisionless, spatial separation of ions of various types occurs (in our case, $B_z = 0.1$ T on the z axis at a plasma density of $n_e \sim 10^{11} \text{ cm}^{-3}$ in the separation area). The components of B_r and B_z determine the specific features of the plasma drift in the cylindrical chamber of the separator. Based on the distribution of the magnetic field lines, in the separation area two areas can be conditionally distinguished (see Fig. 1). In the area 1 magnetized ions and electrons, which are injected from the PS, are transported along the magnetic field lines in the vacuum chamber of the separator. Note that in the area 1 B_r and B_z vary slightly (see the values of B_r in the interval $0 \leq r \leq 25$ cm in Fig. 3,a and B_z in the interval $125 \leq z \leq 250$ cm in Fig. 3,b). The gradient and polarization drift of plasma is absent or not significant. In the area 2 the gradient $\nabla B \neq 0$ and it depends on the magnetic field geometry (see the values of B_r in the interval $25 \leq r \leq 130$ cm in Fig. 3,a and B_z in the interval $125 \leq z \leq 250$ cm in Fig. 3,b). At such conditions

charged plasma particles change velocity, and a local charge in the plasma can occur, resulting in the appearance of electric polarization fields that affect the plasma drift.

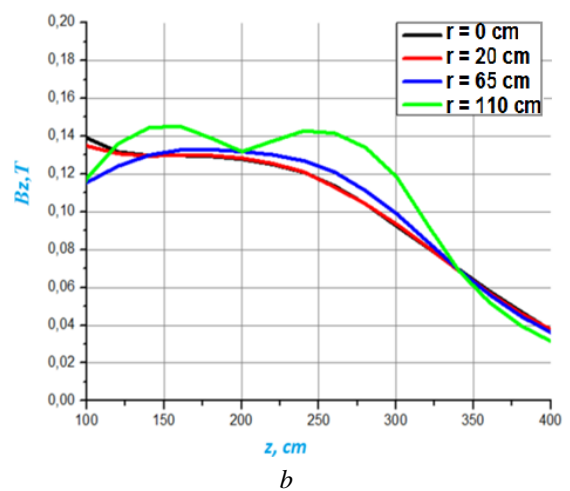
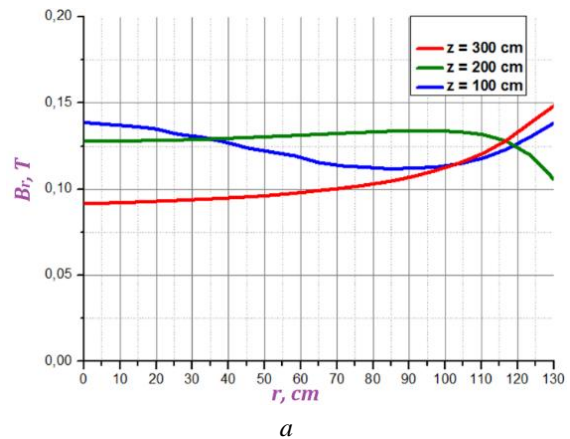


Fig. 3. Distribution of B_r and B_z , respectively, along the radius (a) and length (b) in the DIS setup

In our case, the target ions ($M = 232 \dots 277$) under the action of the electric field E_r [7] are transported in the radial direction to the collector in the area 2 (see Fig. 1). The structure of the electric field in this area will affect the dynamics of the plasma, and the ion trajectories will deviate from the calculated ones in the one-particle approximation [7]. To remove the polarization fields in the separation area, it is necessary to compensate the space charge of the ions by electrons. To solve this problem, it is possible to use electron emitters, located in the area 2, taking into account the equipotentials E_z , which are determined by the superposition of the magnetic fields of the separator coils. In addition, the electrons from the emitter will ionize the residual gases, forming a plasma with a density of $\sim 10^{11} \text{ cm}^{-3}$ in the separator chamber. The parameters of the solenoids from the magnetic system of the plasma separator are presented in the Table. For cooling the superconducting windings in cryostats with liquid helium, it is possible to use, for example, industrial versions of a helium liquefaction plant with a capacity of 40...50 kW [8]. Thus, the energy consumption for creating the magnetic field of the DIS setup by use of solenoids 1–4 is at the level of ~ 250 kW.

Parameters of the DIS solenoids

Solenoid	Current density, A/m ²	Conductor current, A	Number of turns	Resistance, Ω	Winding length, m	Power consumption, kW
1	9·10 ⁷	1350	995	0 (super-conducting mode)	1124	~ 50 (general cryosystem)
2	7·10 ⁷	1050	995	0 (super-conducting mode)	1124	
3	10 ⁶	600	600	0.27	5049	97.2
4	10 ⁶	600	600	0.27	5049	97.2

2. ACTINIDES REMOVAL FROM PLASMA SEPARATOR

To select a given mass range ($M = 232\dots277$) of ions (Fig. 4,a), a variable component of the electric field, E_r , with a frequency equal to half the cyclotron frequency of ions ($M = 238$) is used, which ensures their separation from the ions of another varieties that are deposited on the end collector [9]. The collection of the

target ions can be realized in a localized area of the cylindrical vacuum chamber of the separator, the so-called “pocket”-collector (see Fig. 4,b), wherein apart from UO_2^+ , the ions of other actinides and their oxides can be collected. Ions deposition area is $\sim 15\text{ m}^2$. Inside there are grids with a high transparency coefficient to obtain a uniform distribution of deposited layer onto the collector surface [6].

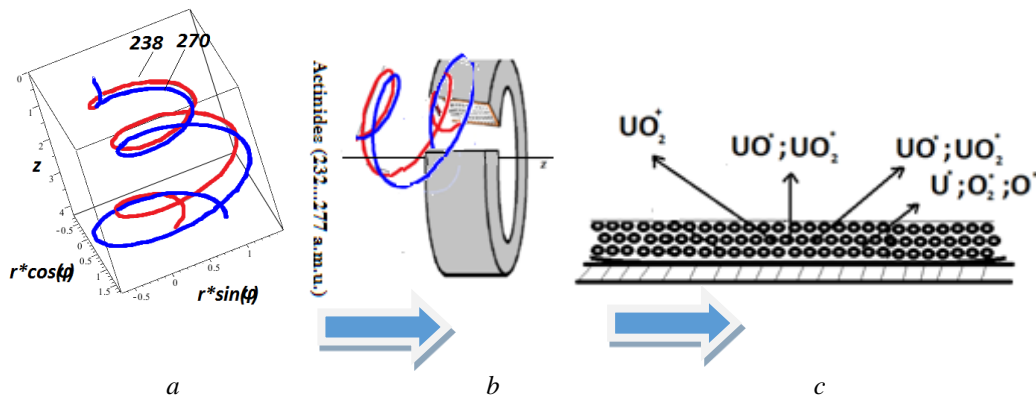


Fig. 4. The sequential process of separation and collection of actinides in the DIS setup: a – 3D trajectories of uranium ions ($M = 238$) and uranium dioxide ions ($M = 270$) in the separator volume; b – motion of ions of a given mass range ($M = 232\dots277$) into the “pocket”-collector; c – deposition of target ions onto the collector

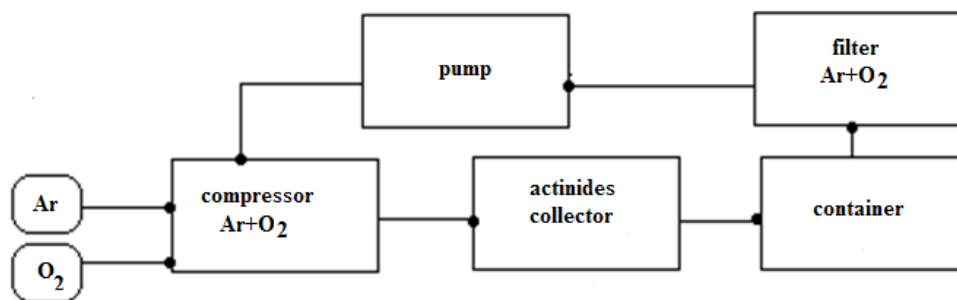


Fig. 5. Procedure of removing actinides from plasma separator to the container

It is expected that the deposition of target ions with an energy of 300...500 eV will be accompanied by sputtering of the deposit layer, formation of a multicomponent plasma and the presence of various plasma-chemical reactions in the “pocket”-collector (see Fig. 4,c).

After deposition of the ions onto the collectors, the plasma separation process ends. Then, a mixture of argon and oxygen is injected in the isolated zone of actinides collection. The converting of the main

component UO_2 into U_3O_8 occurs as a result of chemical reaction



To remove the deposited layer of actinides from the “pocket”-collector, a purge system can be used, which consists of: a subsystem for injecting a mixture of gases of a given type at a temperature of 500...700 °C, a container for accumulating actinide oxides, evacuated pipes, a vacuum pump, vacuum valves, a compressor

and a filter. In the cylindrical vacuum chamber, an opening with a fast-acting vacuum valve is provided for injecting a gas mixture under pressure into the area of the “pocket”-collector. A hole is provided on the inner surface of the “pocket” for removing actinide oxides under the influence of a pressure difference created by a vacuum pump, with the possibility of closing it with a fast-acting vacuum valve. The vacuum valve system ensures the tightness of the system and prevents the penetration of actinides from the “pocket” area.

An inlet of inert carrier gases, for example, argon, in a mixture with oxygen into the “pocket”-collector area, leads to a significant change in the forces that act in the condensed layer, to its cracking, a change in the parameters of the crystal lattice of UO_2 and the converting of UO_2 into U_3O_8 [10], after which the U_3O_8 powder together with other actinide oxides is blown out by the flow of a circulating gas mixture from the collector area into the container. Similarly, together with uranium, other actinides are converted into oxides. This technique could be a useful model for the production of MOX fuel for power reactors.

CONCLUSIONS

The parameters of the magnetic system of the demomitation separator, which consists of a combination of superconducting and water-cooled windings, are presented. The features of plasma drift in a magnetic field of a given geometry are pointed. Two areas are conditionally identified inside the separation area, one of which is associated with the gradient and polarization drift. To compensate the space charge of the ions ($M = 232...277$) flux in this area, it is proposed to use electron emitters, located taking into account the equipotentials of E_z , the distribution of which is determined by the geometry of the magnetic field of the separator.

The estimation of the energy consumption for the creation of the magnetic field of the DIS, which is 250 kW, has been carried out.

A method is proposed for actinides removing from the system using gas technologies, where a mixture of oxygen and argon is injected into the actinide collection area with further removal of the powder mixture of actinide oxides from the plasma separator into an external container.

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ВЫДЕЛЕНИЕ АКТИНОИДОВ В ДЕМОНСТРАЦИОННО-ИМИТАЦИОННОМ СЕПАРАТОРЕ С МАГНИТНЫМ ПОЛЕМ ЗАДАННОЙ КОНФИГУРАЦИИ

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Для моделирования очистки ОЯТ от продуктов деления отрабатывается концепция демонстрационно-имитационного сепаратора с плазмой, вращающейся в скрещенных электрическом и магнитном полях. Представлены параметры магнитной системы, состоящей из комбинации сверхпроводящих и водоохлаждаемых обмоток. Отмечены особенности дрейфа плазмы в магнитном поле заданной геометрии. Проведена оценка энергозатрат на создание магнитного поля сепаратора, которая составляет ~ 250 кВт. Предложен способ выведения актиноидов из плазменного сепаратора во внешний контейнер.

ВИДІЛЕННЯ АКТИНОЇДІВ У ДЕМОНСТРАЦІЙНО-ІМІТАЦІЙНОМУ СЕПАРАТОРІ З МАГНІТНИМ ПОЛЕМ ЗАДАНОЇ КОНФІГУРАЦІЇ

В.Б. Юферов, В.В. Катречко, В.О. Ільчова, С.М. Хіжняк

Для моделювання очищення ВЯП від продуктів поділу відпрацьовується концепція демонстраційно-імітаційного сепаратора з плазмою, що обертається в схрещених електричному та магнітному полях. Представлені параметри магнітної системи, яка складається з комбінації надпровідних обмоток й обмоток, що охолоджуються водою. Відзначено особливості дрейфу плазми в магнітному полі заданої геометрії. Проведена оцінка енергозатрат на створення магнітного поля сепаратора, яка складає ~ 250 кВт. Запропоновано спосіб виведення актиноїдів з плазмового сепаратора в зовнішній контейнер.